

# Neutron induced reaction and neutron sources

## Introduction to Nuclear Science

Simon Fraser University  
SPRING 2011

NUCS 342 — April 6, 2011



# Outline

## 1 Neutron-induced reactions

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- 2 Radionuclide neutron sources

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- 3 Accelerator neutron sources
- 4 The SIMON project at SFU

# Nuclear reactions on Earth

- Chemical reactions are commonly observed in the Earth environment.
- There are, however, very few nuclear reactions occurring on Earth.
- There are two reasons accounting for that fact
  - Nuclear reactions between charged ions (for example proton-induced reactions) need to overcome the Coulomb barrier. There is not enough thermal energy at standard temperatures to achieve that.
  - Neutrons, which can penetrate into a nucleus without the Coulomb barrier at standard temperatures, have 614 s ( $\sim 10$  min) half-life and are present in the environment with extremely low quantities.
- Nuclear reactions induced by cosmic rays in the upper parts of the atmosphere are the source of radio nuclides ( $^{14}\text{C}$ ) in the environment.
- Nuclear decays are commonly observed.

# Man-made sources of neutrons

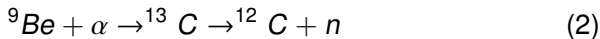
- The lack of Coulomb barrier makes neutrons very attractive tool for nuclear transmutation.
- Short neutron lifetime prompted development of man made sources of neutrons.
- These sources fall into four general categories:
  - Neutrons can be produced by nuclear reactions induced by  $\alpha$  particles from naturally occurring  $\alpha$  emitters.
  - Neutrons can be produced by nuclear reaction induced by accelerated beams of light ions.
  - Neutrons can be produced from a spallation reaction.
  - Neutrons can be produced from fission.
- These different categories provide sources of various characteristics and can be selected and tuned for specific applications.

## Pu-Be neutron source

- Let us calculate the Coulomb barrier for the reaction of  $\alpha$  particles on beryllium

$$V_C = 1.44 \frac{2 \cdot 4}{1.2(\sqrt[3]{9} + \sqrt[3]{4})} = 2.6 \text{ [MeV]} \quad (1)$$

- This implies that for most of naturally occurring  $\alpha$  emitters the energy of  $\alpha$  particles is high enough to induce nuclear transmutation of beryllium.
- Such mixtures are useful neutron sources since



- A popular source is a mixture of  ${}^{241}\text{Pu}$  with  ${}^9\text{Be}$ .
- It produces 30 neutrons per million of emitted  $\alpha$  particles.



# Radionuclide neutron sources

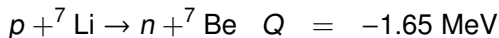
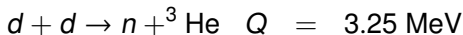
- Other radionuclide neutron sources are

Radionuclide	$T_{1/2}$	Neutron Yield [n/Ci]
$^{210}\text{Po}$	138 d	$2.5 \times 10^6$
$^{238}\text{Pu}$	87.8 y	$2.2 \times 10^6$
$^{241}\text{Am}$	433 y	$2.2 \times 10^6$
$^{242}\text{Cm}$	163 d	$2.5 \times 10^6$
$^{252}\text{Cf}$	2.65 y	$4.3 \times 10^9$

- The decay rate conversion is  $1 \text{ Ci} = 3.7 \times 10^{10} \text{ Bq} = 37 \text{ billion decays per second}$ .
- The emission of neutrons is isotropic, meaning has equal probability in any direction.

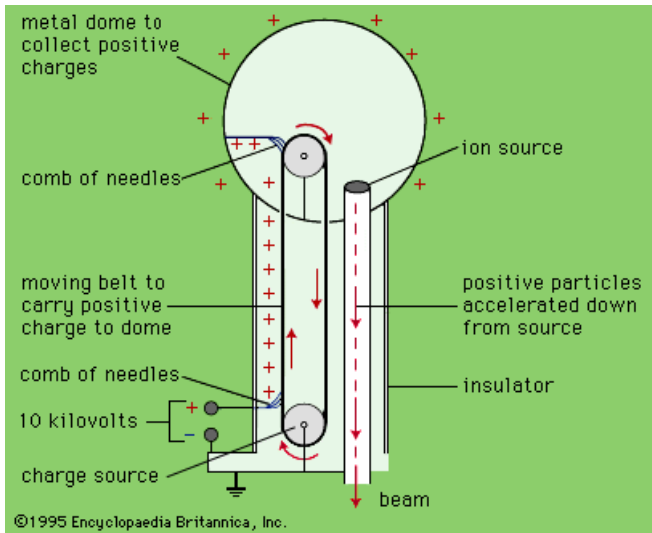
## Light ion reactions producing neutrons

- A number of nuclear reactions producing neutrons in the final state have been identified.
- The most common are

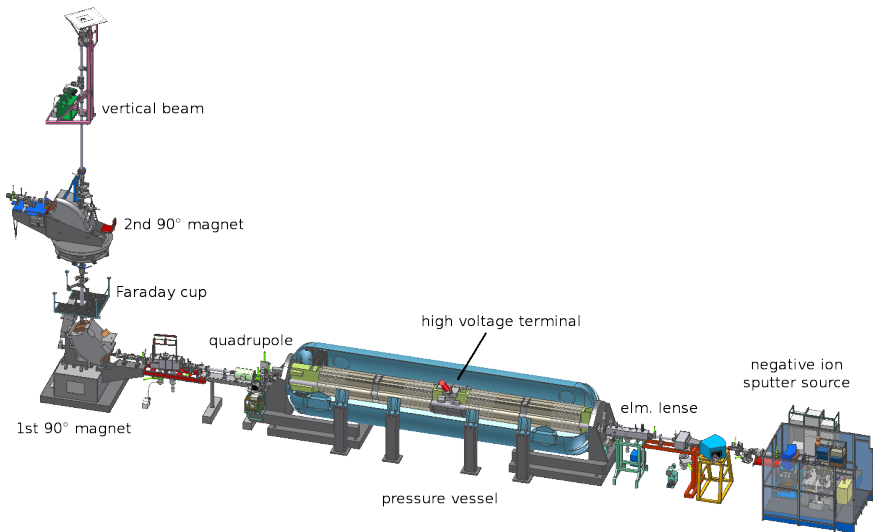


- The light ion beams are accelerated to the energies slightly above the Coulomb barrier by charged particle accelerators.
- Reaction kinematics can be used to tune the outgoing neutron energy based on the energy of the incoming beam.

# Van de Graaff accelerator

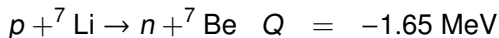


# Tandem Van de Graaff accelerator



# Reaction kinematics and neutron energy

- Let us consider the reaction



- For the neutrons emitted at zero degree

$$\vec{p}_p = \vec{p}_n + \vec{p}_{7\text{Be}}$$

$$\vec{p}_{7\text{Be}} = \vec{p}_n - \vec{p}_p$$

$$p_{7\text{Be}}^2 = p_n^2 + p_p^2 - 2\sqrt{p_p p_n}$$

- But the energy momentum relation gives

$$T = \frac{p^2}{2m} \implies p^2 = 2Tm \quad (3)$$

- Which leads to

$$7T_{7\text{Be}} = T_n + T_p - 2\sqrt{T_p T_n} \quad (4)$$

## Reaction kinematics and neutron energy

- The  $Q$  value definition gives

$$Q = T_{7Be} + T_n - T_p \quad (5)$$

- Combining the above with

$$7T_{7Be} = T_n + T_p - 2\sqrt{T_p T_n} \quad (6)$$

leads to

$$\begin{aligned} 7(Q - T_n + T_p) &= T_n + T_p - 2\sqrt{T_p T_n} \\ 8T_n - 6T_p - 2\sqrt{T_p T_n} &= 7Q = -11.52 \text{ MeV} \end{aligned} \quad (7)$$

- The solution of the above equation gives energy of the mono-energetic neutron beam as a function of the incoming proton energy.

# Properties of neutrons from accelerator sources

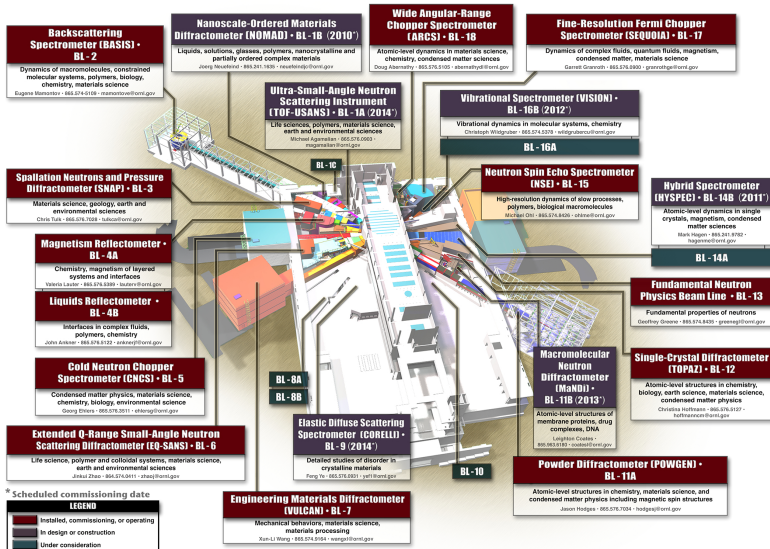
- Emission of neutrons from accelerator sources is anisotropic (not isotropic).
- Reaction kinematics focuses neutrons in the direction of the incoming beam.
- On the top of that there is a correlation between the direction of a neutron and its energy, larger angles give smaller energy neutrons.
- Neutron beams can be provided using collimation.
- Neutron beam energies can be tuned using the energy of the incoming ions and energy-angle correlation.
- Neutron beams are used for basic and applied research, in particular, studies of material properties.

# The Spallation Neutron Source

- The Spallation Neutron Source is an accelerator-based neutron source that provides the most intense pulsed neutron beams in the world for scientific and industrial research and development.
- It is located in Oak Ridge Tennessee in US.
- The accelerator is 1 GeV proton linear accelerator.
- The beam is pulsed with the bunch time width of 1  $\mu$ s.
- The spallation target is liquid mercury.
- 20-30 neutrons are emitted per spallation.
- Neutron beams are formed by slowing down in moderators and by collimation.



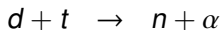
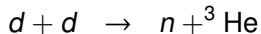
# Spallation Neutron Source Instruments



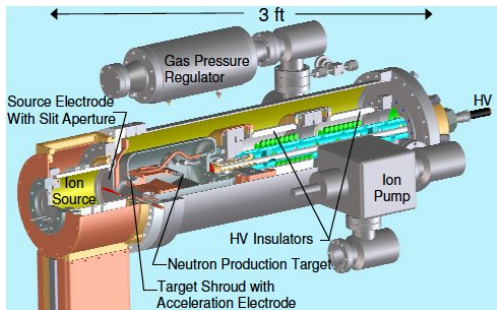
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# Sealed tube neutron generators

- Sealed tube neutron generators are small accelerators which generate neutrons via



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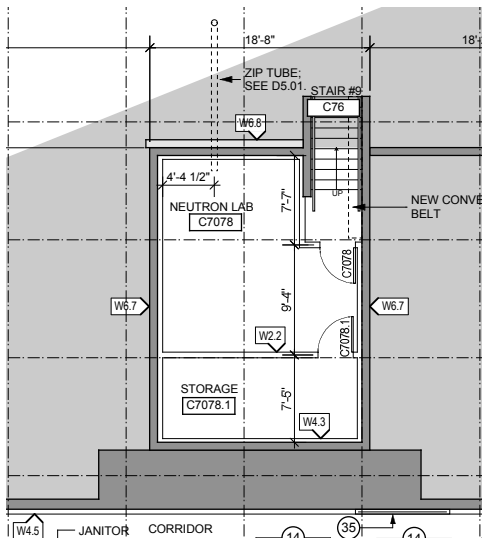
## Sealed tube neutron generators

- Sealed tube neutron generators are portable neutron sources.
- They generate quasimonoenergetic beams of neutrons with 14.1 MeV energy for the  $d/t$  and 2.5 MeV energy for the  $d/d$ .
- Neutrons from the  $d/t$  generator are emitted isotropically, while for the  $d/d$  generator the emission is slightly focused towards the beam direction.
- Yields are on the order of  $10^8$  neutrons/s for the sealed tube  $d/t$  generators, a factor of 50-100 smaller for the  $d/d$  generators.
- The use of radioactive tritium is a concern in application of  $d/t$  generators, this is a reason for the sealed tube solution.
- Other technical implementations of  $d/t$  neutron generator can reach fluxes of  $10^{11}$  particles per second.

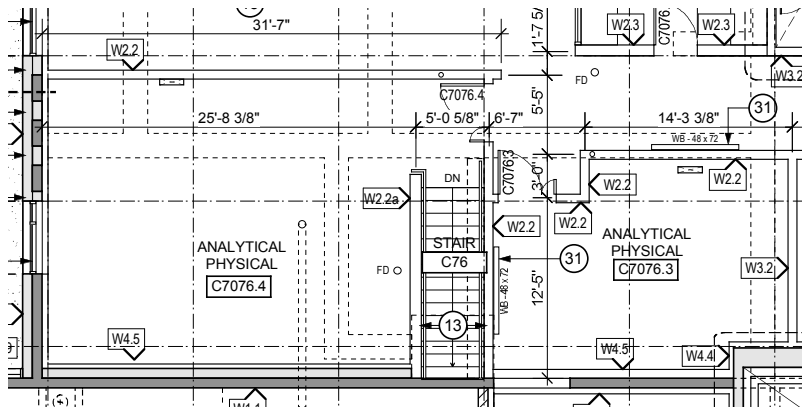
## Nuclear Science facilities

- The SFU's Chemistry building is renovated as a part of the \$50M Chemistry building renovation funded by the provincial government and Industry Canada's Knowledge Infrastructure Program.
- Renovation will be completed in the spring of 2011, new laboratories will be operational in the summer of 2011.
- Nuclear Science facilities of ~\$750,000 value will include
  - A renovated underground radiation vault, ready to host a D/T or D/D neutron generator.
  - A radio-chemistry laboratory above the vault, with a fume-hood, glove-box and other equipment set up for chemical reprocessing of radioactive materials.
  - Conduits between the radiation vault and the radio-chemistry lab designed to accommodate gas jet and a pneumatic system (a rabbit) for transportation of radioactive samples.
  - A separate laboratory for detector development and testing.

# The radiation vault



# The Nuclear Science Laboratories



# The current status



# The SIMON project

- The role of SIMON is to provide access to high intensity neutron beams without a nuclear reactor or spallation source.
- SIMON will comprise of
  - D/T or D/D generator of fast neutrons
  - Beryllium shroud multiplying fast neutrons through the (n,2n) reaction
  - a Heavy-Water moderator
  - low-enriched Uranium for multiplication of moderated neutrons through the neutron-induced fission.
- The goal is to develop a medium-size device (less than 7 m long, 3 m diameter), fitting the floor-plan of the radiation vault at SFU.
- The achievable flux is being examined, the target is  $10^{13}$  n/cm<sup>2</sup>/s.

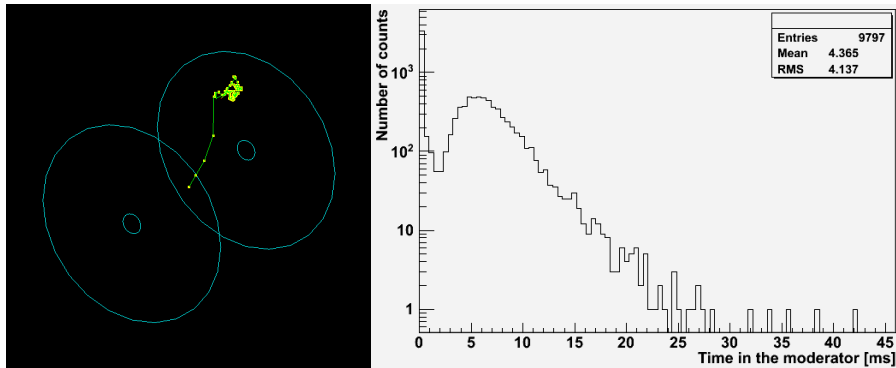


# The road map for SIMON: 2011

- A \$150K Canadian Foundation of Innovation/ British Columbia Knowledge Development Fund proposal for a commercial neutron generator has been awarded
- GEANT4 Monte-Carlo simulation codes for the neutron moderator and multipliers developed based on the high-precision models available from GEANT4 for  $E < 20$  MeV neutrons.
- Optimization of the moderator and multiplier geometry based on the GEANT4 simulations is currently pursued.
- A  $3 \times 10^8$  commercial generator will be acquired.
- Licensing process for the renovated vault and a generator to be acquired will be initiated in early 2011.
- Funds for construction of a test moderator will be sought.

# Moderation of neutrons in GEANT4

(Left) Simulated random walk, (Right) Simulated time in the moderator.



Simulated average time in a heavy-water moderator is  $\sim 4$  [ms].

## SIMON: back on the envelope estimates

- The volume of 100 [cm] long moderator with 50 [cm] radius and a bore for inserting the generator with 5 [cm] radius is

$$V = 100 * \pi * (50^2 - 5^2) \text{ [cm}^3\text{]} = 7.8 \times 10^5 \text{ [cm}^3\text{]}$$

- Number of neutrons in the moderator is equal to the generator output  $f$  times the lifetime in the moderator  $\tau$

$$n = f\tau = 3 \times 10^8 \text{ [part./s]} * 4 \times 10^{-3} \text{ [s]} = 1.2 \times 10^6 \text{ [part.]}$$

- Number density of the neutrons in the moderator is

$$N = \frac{n}{V} = \frac{1.2 \times 10^6}{7.8 \times 10^5} = 1.5 \text{ [part./cm}^3\text{]}$$

## SIMON: back on the envelope estimates

- Assuming thermal neutrons with energy 0.025 [eV] and speed  $v$  of 2.2 [km/s]= $2.2 \times 10^5$  [cm/s] the neutron flux in the moderator without any neutron multiplication is

$$\phi = N * v = 1.5 * 2.2 \times 10^5 \text{ [part./cm}^2\text{/s]} = 3.3 \times 10^5 \text{ [part./cm}^2\text{/s]}$$

- If there is a reaction in the moderator which contributes  $\Delta n$  neutrons per second into the full moderator the number density in the moderator will grow in a unit time according to

$$N + \Delta N = N + \frac{\Delta n \tau}{V} = N \left( 1 + \frac{\Delta n \tau}{NV} \right) = N(1 + \lambda) \quad \text{with} \quad \lambda = \frac{\Delta n \tau}{NV}$$

- Growth in time is described by

$$N(t) = N(0)(1 + \lambda)^t$$

## SIMON: back on the envelope estimates

- Let us assume thermal neutron induced fission of natural Uranium as the multiplication reaction.
- Number of thermal-neutron induced fission per second is

$$\Delta n = \phi \kappa a d \bar{\nu} \sigma \frac{\rho}{\mu} N_A = N v \kappa a d \bar{\nu} \sigma \frac{\rho}{\mu} N_A$$

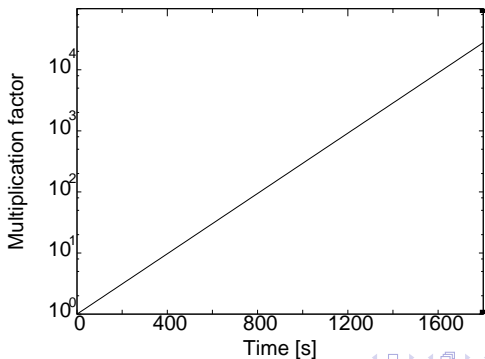
with

- $\kappa = 0.007$  being  $^{235}\text{U}$  content in  $^{\text{nat.}}\text{U}$ ,
- $a$  and  $d$  being  $^{\text{nat.}}\text{U}$  target area and thickness, respectively, with the  $ad = 60 [\text{cm}^3]$  representing the volume of  $^{\text{nat.}}\text{U}$  in the moderator in 0.5 cm thick, 2 cm long bars, 10 cm from the centre,
- $\bar{\nu} = 2.5$  being the average number of neutrons per  $^{235}\text{U}$  fission,
- $\sigma = 600 [\text{b}] = 6 \times 10^{-22} [\text{cm}^2]$  is the cross section for 0.025 [eV] thermal neutron capture on  $^{235}\text{U}$ ,
- $\rho = 19 [\text{g}/\text{cm}^3]$  is the density of  $^{\text{nat.}}\text{U}$ ,
- $\mu = 239 [\text{g}/\text{mol}]$  is the molar mass of  $^{\text{nat.}}\text{U}$ ,
- $N_A = 6 \times 10^{23}$  is the Avogadro number.

## SIMON: back on the envelope estimates

- Combining above equations the multiplication factor for 800 kg of heavy water and 1.2 kg of  $^{235}\text{U}$  becomes

$$F(t) = \frac{N(t)}{N(0)} = (1 + \lambda)^t \quad \text{with} \quad \lambda = \kappa \bar{v} \frac{ad}{V} v \tau \sigma \frac{\rho}{\mu} N_A = 5.7 \times 10^{-3}$$



## The road map for SIMON: long-range

- GEANT4 codes will be validated using results obtained with the low-flux generator and the test moderator.
- Working parameters for the final neutron generator and the moderator will be specified and the final design will be defined based on the validated calculation.
- Nuclear engineering assistance will be sought in regard to shielding, design, manufacturing and operation of the final neutron generator and moderator assembly.
- Utility of the SIMON for fundamental and applied research program will be examined.

# The utility of SIMON

- SIMON is a project with significant discovery potential
  - Can become a driver for production of rare isotopes with large neutron excess via neutron-induced fission.
  - Consequently, it can support state of the art nuclear research probing the beginning of the Universe and answering key fundamental scientific questions regarding nucleosynthesis and distribution of elements.
- SIMON provides an ideal training opportunities for future generation of Highly Qualified Personnel in Nuclear Science.
- SIMON may open a way to produce important radioisotopes for medical and commercial applications without use of reactors.
- For material sciences SIMON will enable neutron activation analysis providing information on elemental composition on a particle-per-billion level in a non-destructive way.
- SIMON may also become a tool for neutron scattering aiding research in material sciences.